

Computational neutronics analyses of deuteron interactions with lithium target in IFMIF-DONES for fusion applications



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Content



- **Part I:** Basic processes defined for (d-Li) atomic and nuclear interactions. Illustration on a simple model of cylindrical solid Li inside the aluminum capsule.
- **Part II:** Application of the d-Li accelerator-based intense neutron source of IFMIF-DONES for fusion applications.
- **Conclusions**
- Backup slides:
 - McDeLicious code parallelization on Marconi-Fusion HPC
 - Use of On-The-Fly (OTF) Monte Carlo variance reduction technique

Radiation transport with the MCNP code



MCNP is a code for radiation transport calculations in 3D geometry. The abbreviation is translated as **M**onte **C**arlo **N**-**P**article.

Neutron, photon, electron, or coupled neutron/photon/electron transport can be performed by MCNP. The MCNP code was developed by X-5 Monte Carlo Team in Los Alamos National Lab. (LANL), USA.

History of the MCNP code development [1]



Contributors to MCNP6.2		
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R. Arthur Forster ¹	Anthony Zukaitis ¹	Michael R. James ⁵
John T. Goorley ¹	Casey Anderson ²	Michael L. Fensin ⁶
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⁶ W-13 Advanced Engineering Analysis, Los Alamos National Laboratory
⁷ XTD--SS Safety & Surety, Los Alamos National Laboratory
⁸ XCP-3 Monte Carlo Methods, Codes, and Applications, Los Alamos National Laboratory, Guest Scientist

Reference:

[1] Avneet Sood, 2017. The Monte Carlo Method and MCNP-A Brief Review of Our 40 Year History, Presentation to the International Topical Meeting on Industrial Radiation and Radioisotope Measurement Applications Conference.

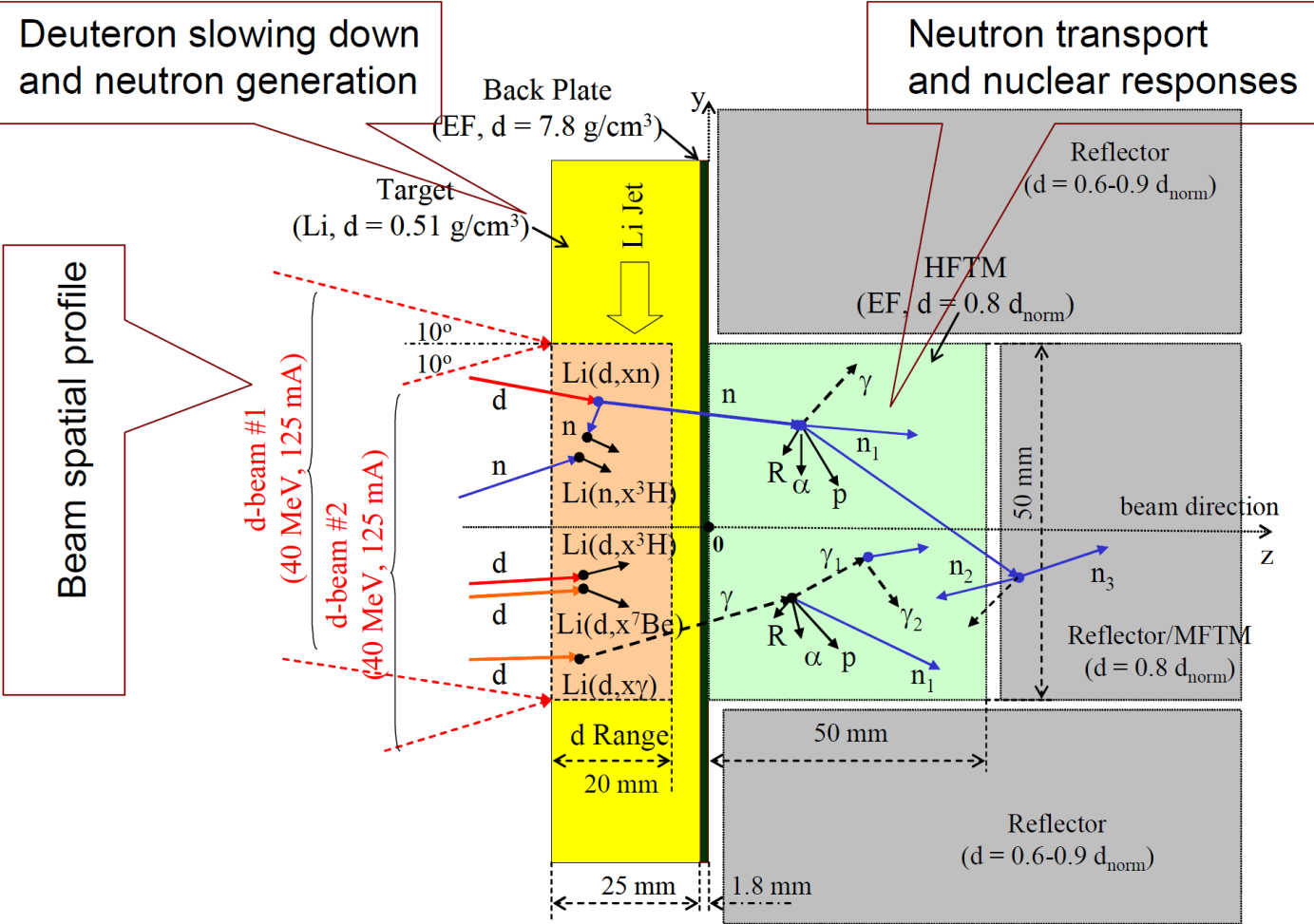
McDeLicious is an extension to the **MCNP Monte Carlo** code with the ability to simulate the generation of source neutrons based on deuteron - lithium (**D-Li**) interaction processes

- 1999: **McDeLi** (*P. Wilson, Report FZKA 6218, 1999*):
 - An enhancement to MCNP-4a to sample the generation of d-Li source neutrons based on embedded analytical formulas representing direct deuteron stripping (Serber model) and compound reactions.
- 2001: **McDeLicious** (*S.P.Simakov et al. J.Nucl.Mat.307-311(2002)1710, FZKA 6743*)
 - An enhancement to MCNP-4b,c to sample the d-Li source neutrons on the basis of tabulated double-differential d + ${}^6,7\text{Li}$ cross-sections for deuteron energies up to 50 MeV (evaluated by *A. Konobeyev et al., NSE 139 (2001)1*).
- 2005: **McDeLicious-05** – compilation with MCNP-5 and use tabulated double-differential cross-sections from updated d + ${}^6,7\text{Li}$ evaluation (*made by P. Pereslavytsev et al., J.Nucl.Mat.367-370(2007)1531*).
- 2011: **McDeLicious-11** - a new approach is implemented to enable direct sampling from the tabulated deuteron beam distribution data without using fitting functions. In this approach, the beam entry position is sampled from tabulated data representing the intensity distribution of the impinging deuteron beam – (*S. P. Simakov et al., "Status of the McDeLicious approach for the D-Li neutron source term modeling in IFMIF neutronics calculations," Fusion Sci. Technol., 62 (2012), pp. 233-239*)
- 2017: **McDeLicious-17** – the actual version of McDeLicious upgraded to MCNP version 6.1.0, an extension of the MCNP Monte Carlo code with the capability to simulate the deuterium-lithium neutron source on the basis of evaluated d + ${}^6,7\text{Li}$ cross-section data. This code has been tested and confirmed to generate identical source particle data as the previous version McDeLicious-11 – (*Y. Qiu et al., "IFMIF-DONES HFTM neutronics modeling and nuclear response analyses," Nuclear Materials and Energy, 15 (2018), pp. 185-189*)

Part I

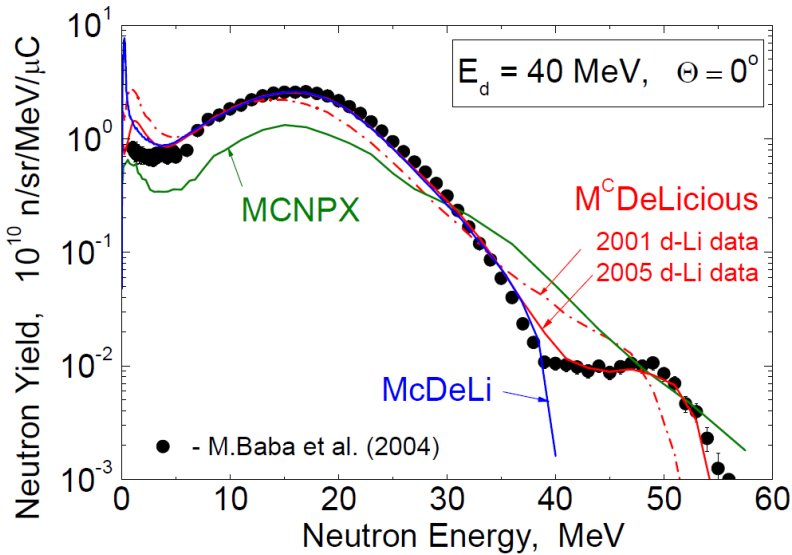
Basic processes defined for (d-Li) atomic and nuclear interactions. Illustration on a simple model of cylindrical solid Li inside the aluminum capsule

Basic nuclear processes in IFMIF

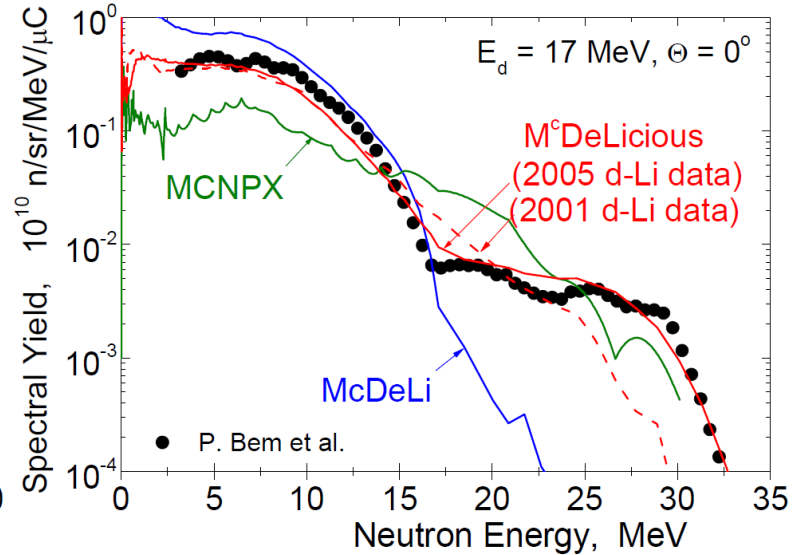


Thick Li-target neutron source: Energy-Angular Yield

$E_d = 40$ MeV
Exp.: M. Hagiwara et al.



$E_d = 17$ MeV
Exp.: P. Bem et al.



Simple Model: cylindrical solid Li inside Aluminum capsule

```
probid = 09/02/24 04:53:10  
basis: XY  
< 1.000000, 0.000000, 0.000000  
< 0.000000, 1.000000, 0.000000  
origin:  
< 1.25, 0.00, 0.00  
extent = < 2.00, 2.00  
cell labels are  
cell names
```

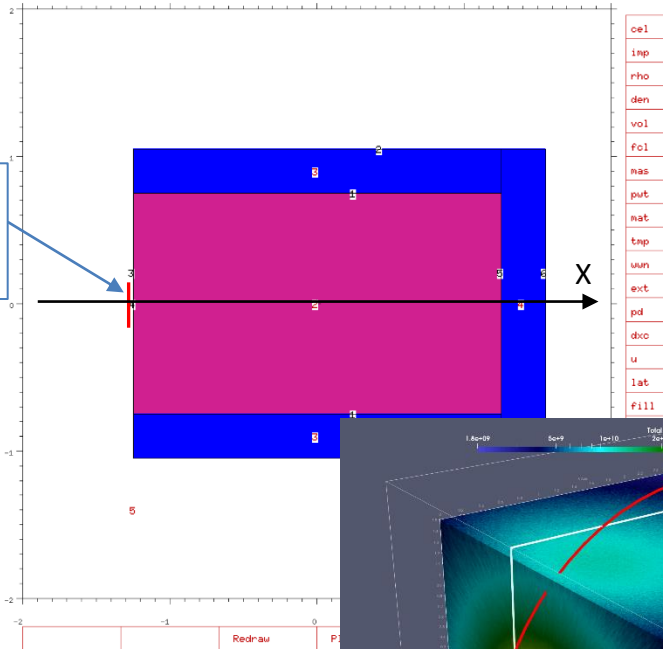
Disk D = 3 mm
D+ source
Ed = 40 MeV
Id = 1 microA

Value for cell 2

In Cell 2
xyz = 1.25, 0.00, 0.00

CURSOR	Restore	CellLine
PostScript	ROTATE	
COLOR	SCALES 1	LEVEL
XY	YZ	ZX
LABELS	L1 sur	L2 cell
HBODY		LEGEND off

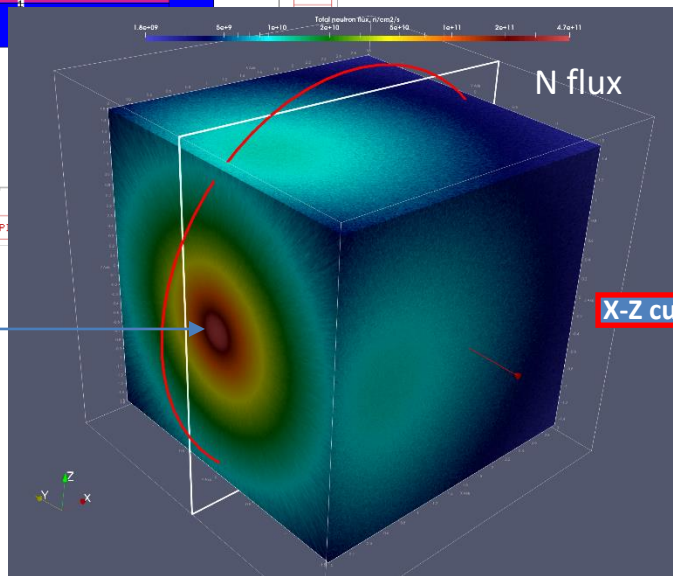
MCNP model



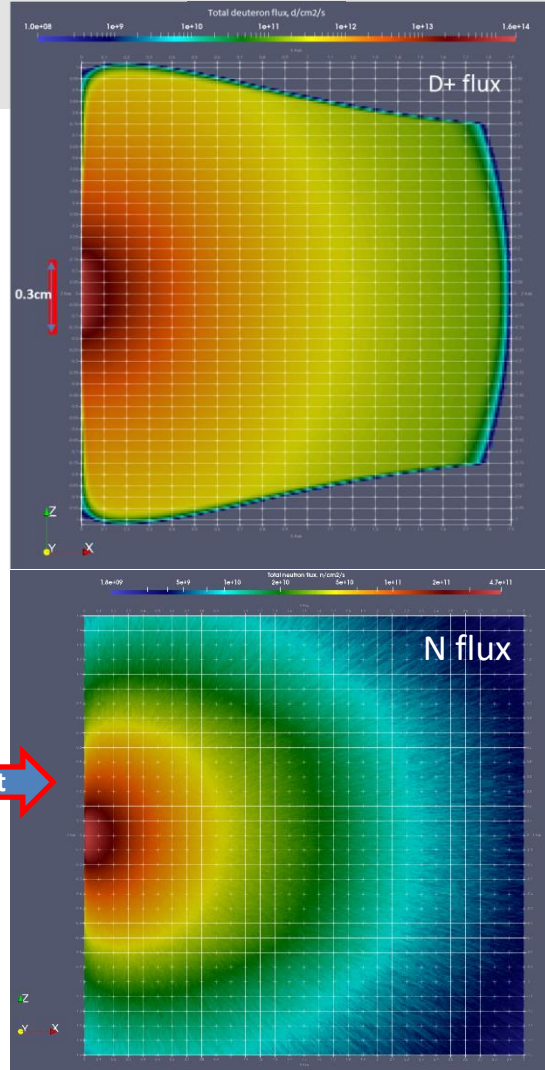
cel
inp
rho
den
vol
fol
nas
put
nat
tsp
uum
ext
pd
dxc
u
lat
fill

V1: Isotropic
uniformly distributed
disk D+ source

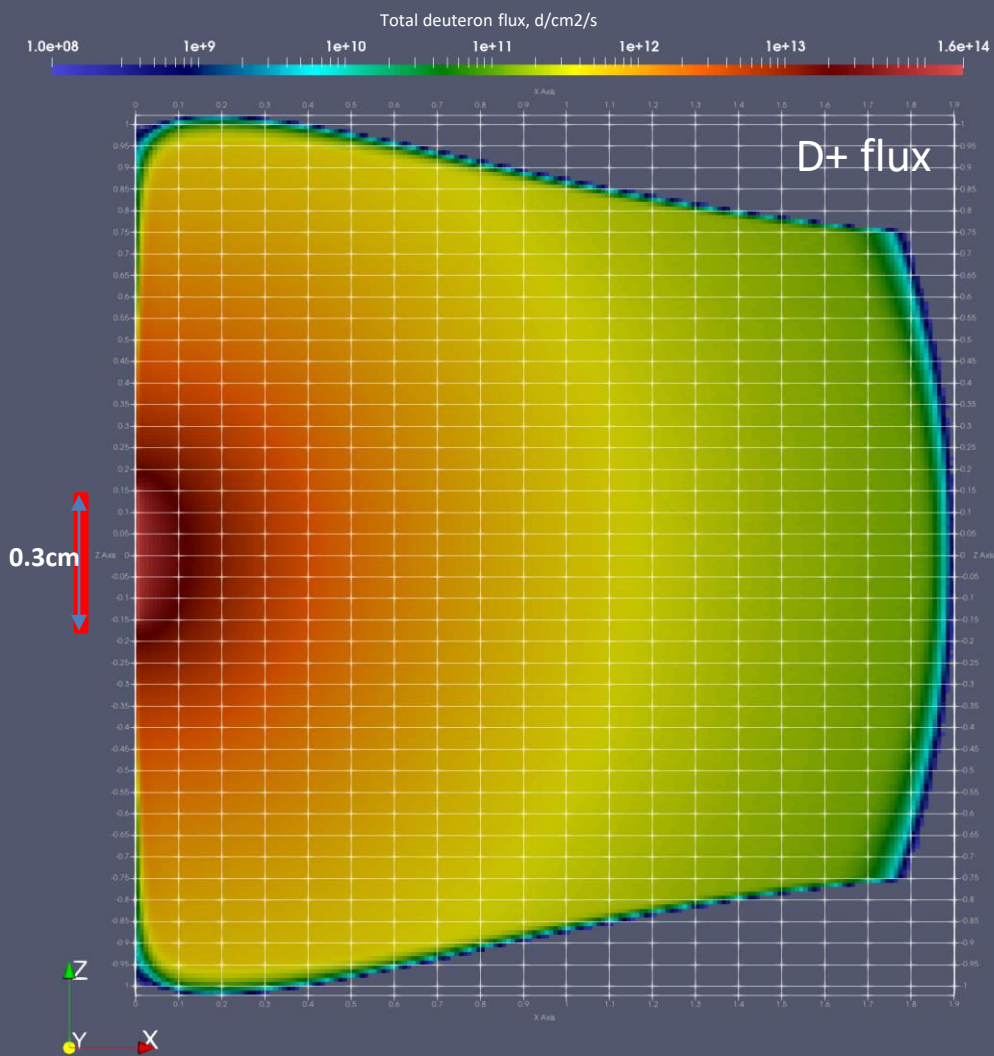
Disk D = 3 mm
D+ source
Ed = 40 MeV
Id = 1 microA



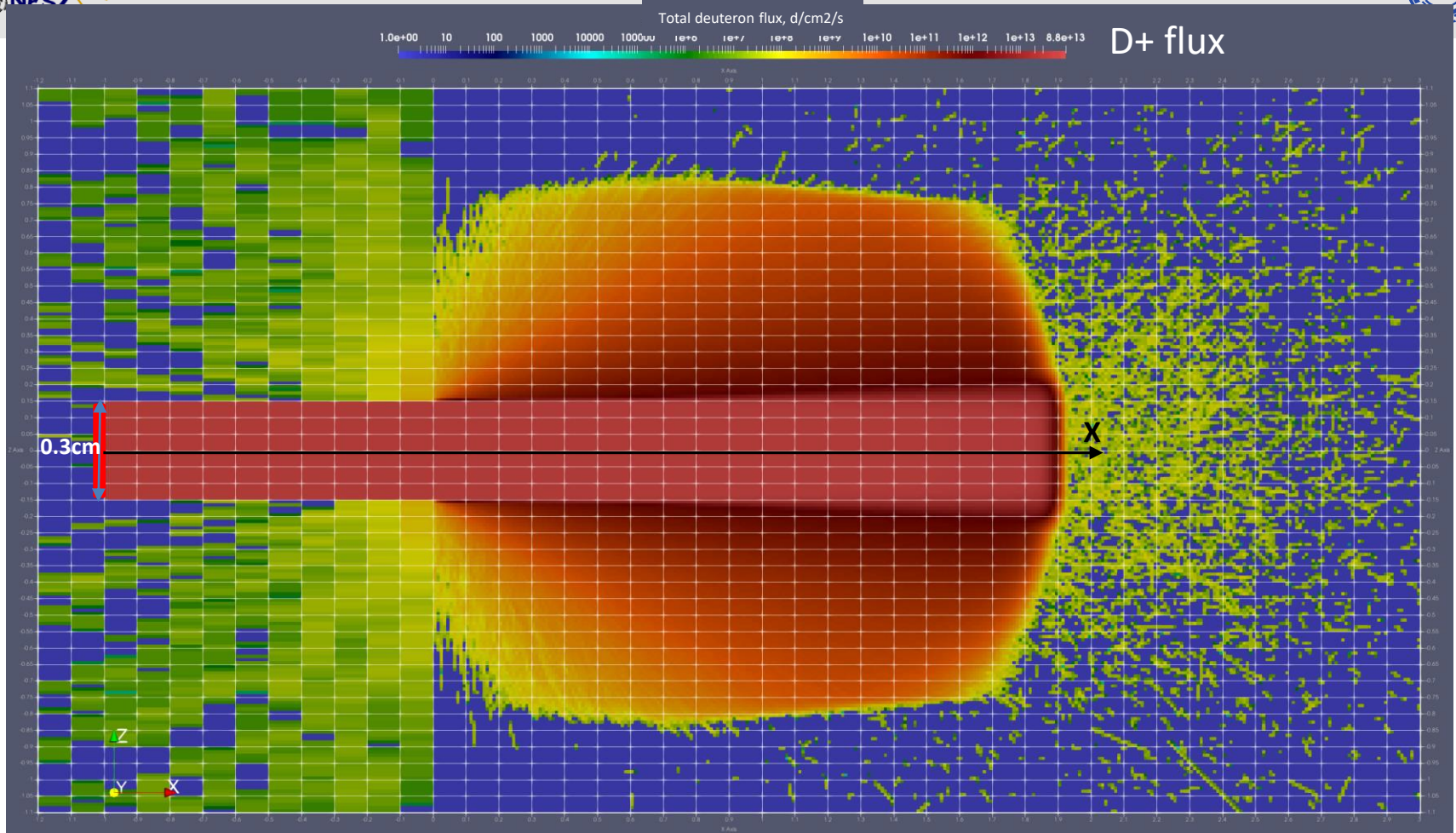
X-Z cut



V1: Isotropic
uniformly distributed
disk D+ source



V2: Monodirectional along X-axis disk D+ source, $E_d = 40$ MeV, $I_d = 1$ microA

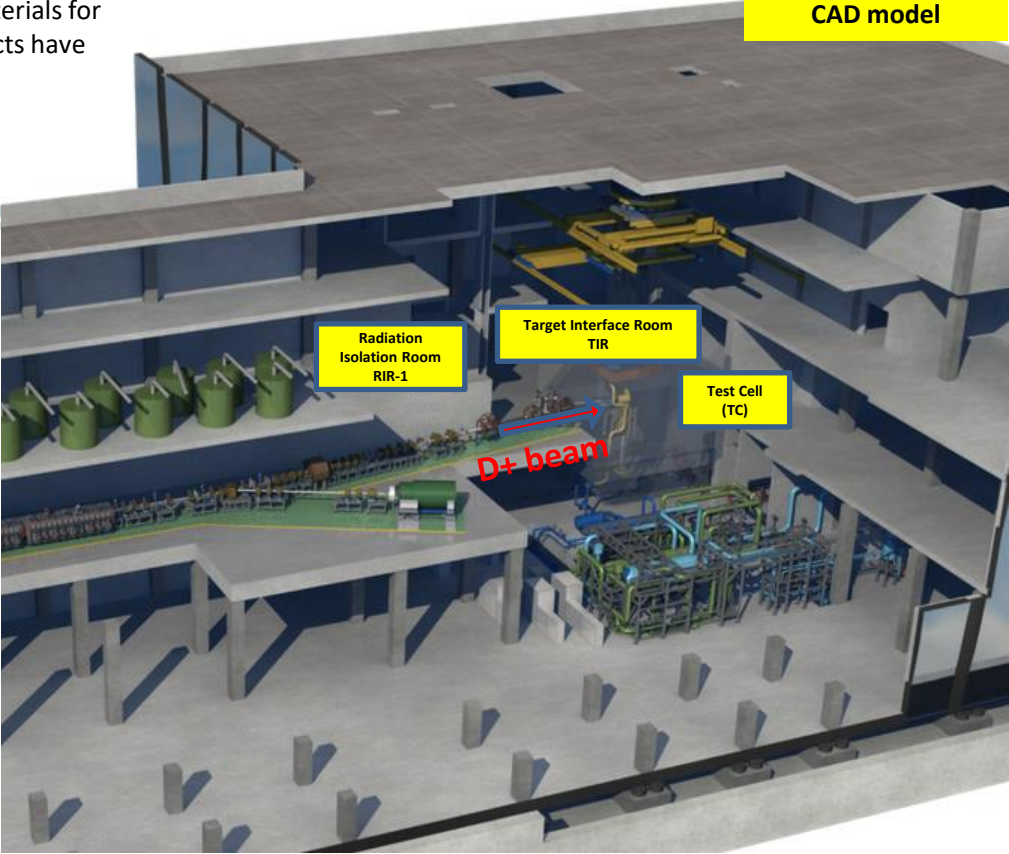
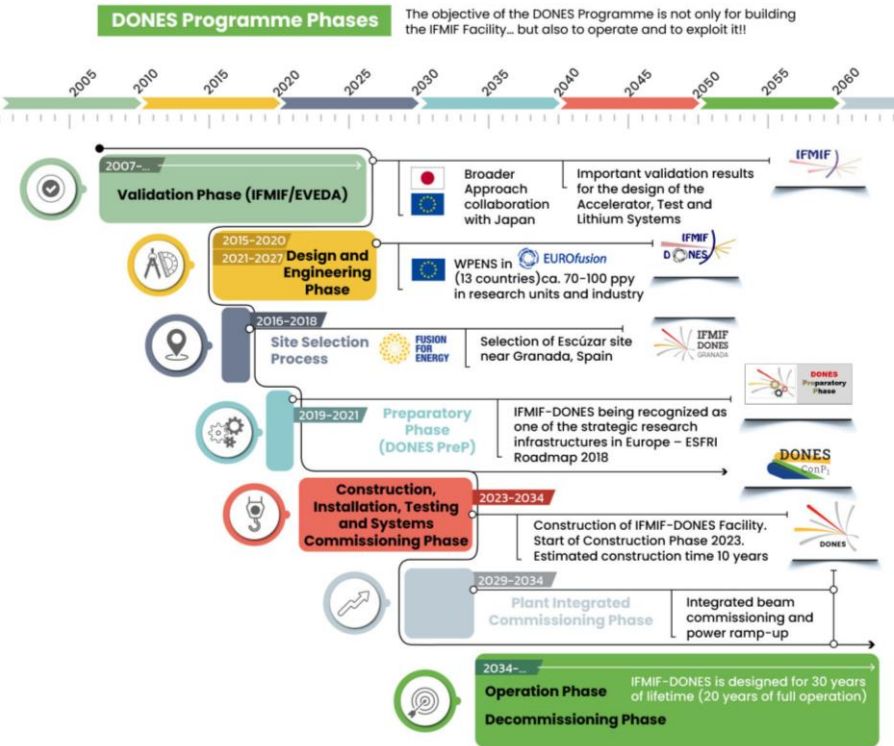


Part II

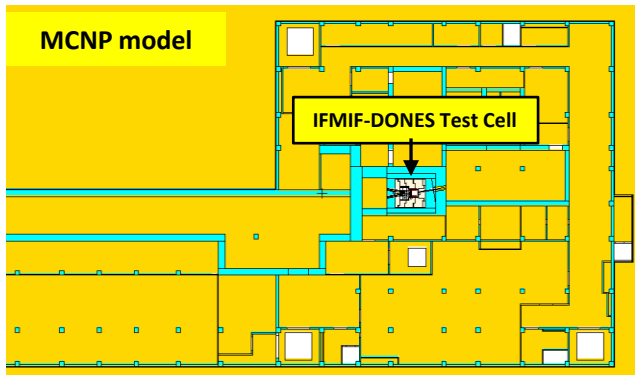
Application of the d-Li accelerator-based intense neutron source of IFMIF-DONES for fusion applications

DONES building CAD model

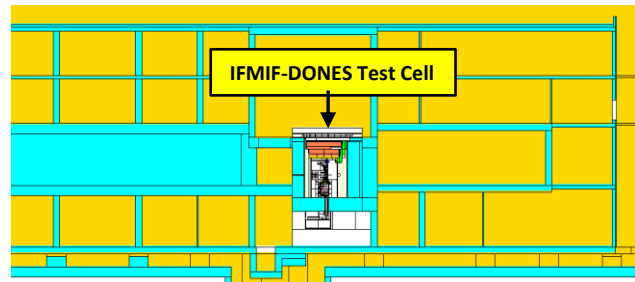
The International Fusion Materials Irradiation Facility—DEMO Oriented NEutron Source (IFMIF-DONES) aims to evaluate and validate the structural and functional materials for developing DEMO-type reactors. To achieve this ambitious goal, several projects have been promoted in recent years, which together form the DONES Programme.



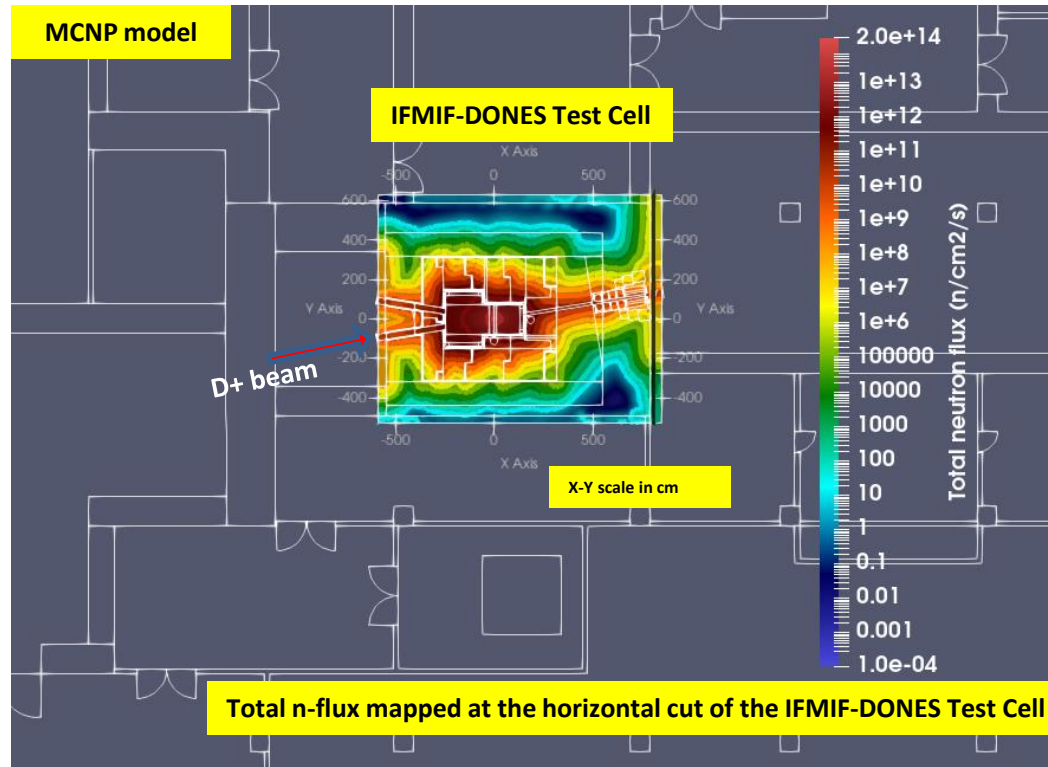
- The CAD model of IFMIF-DONES building is prepared (simplified and decomposed) for the CAD-to-MCNP conversion using the codes: **McCad (INR-KIT developed) or SuperMC (developed by FDS-team, China)**
- **McDeLicious-17 code package developed at INR-KIT** – an MCNP6 code modification for deuteron-lithium (d-Li) nuclear reactions in Li of IFMIF-DONES Test Cell. The beam of deuteron ions accelerated up to 40 MeV with current of 125 mA impinges the liquid Li target delivering 5 MW power. The Li target volume is $5 \times 20 \times 2.5 \text{ cm}^3$



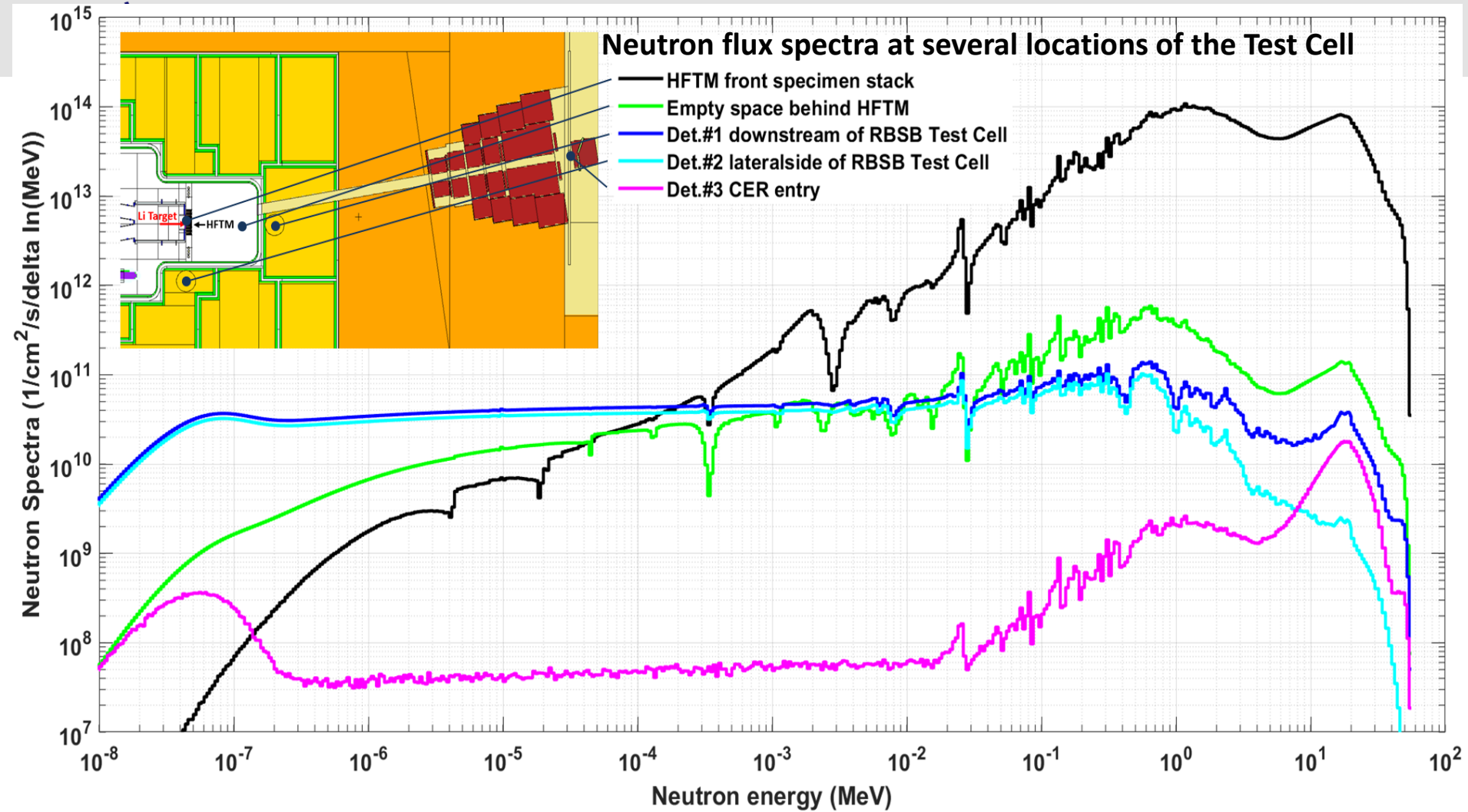
DONES building model horizontal cut at the beam level.

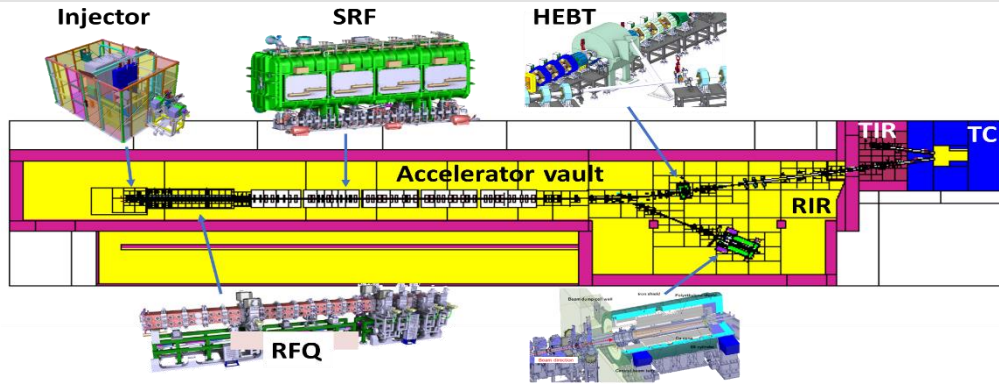


DONES building model vertical cut at the target center



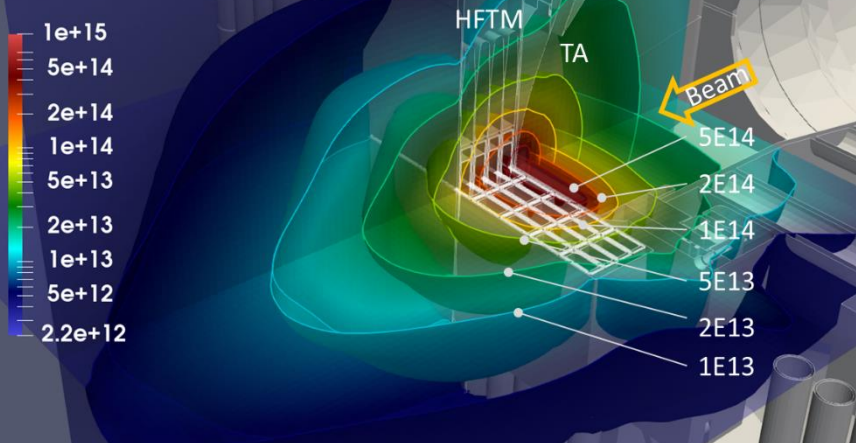
Neutron flux spectra at several locations of the Test Cell





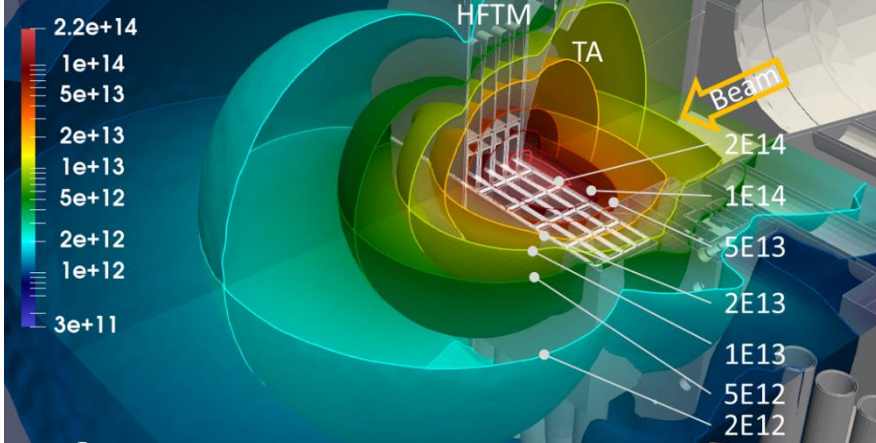
The test cell (TC), which houses the key components of the Target Assembly (TA) and the High Flux Test Module (HFTM)

Neutron flux (n/cm²/s) at Target Assembly (TA)



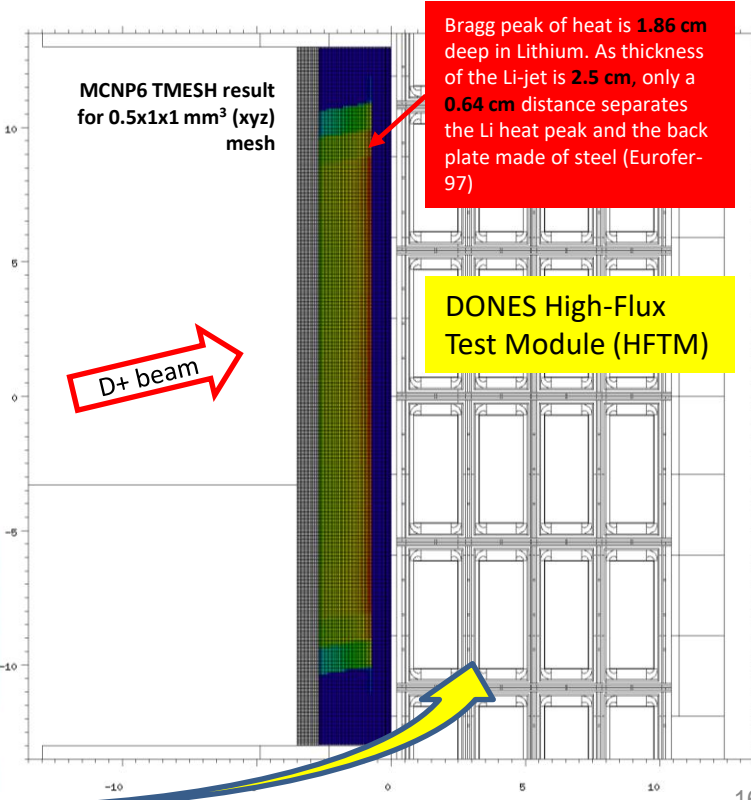
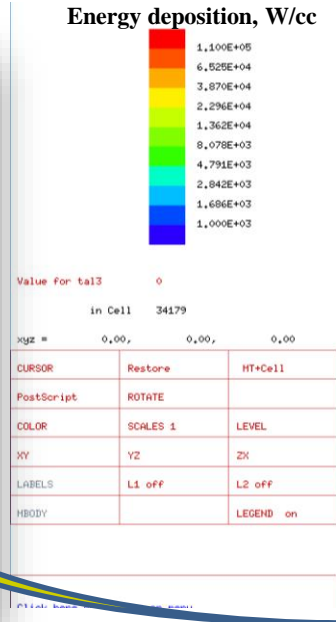
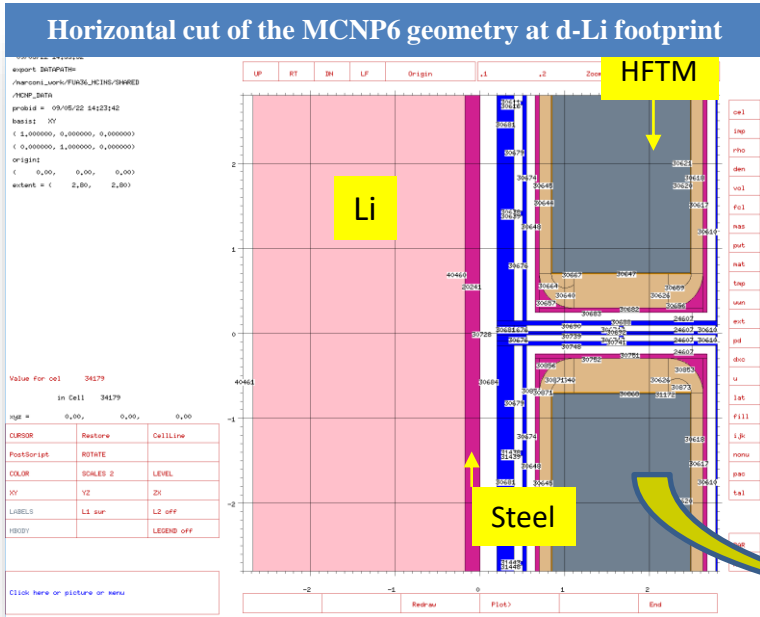
HPBD

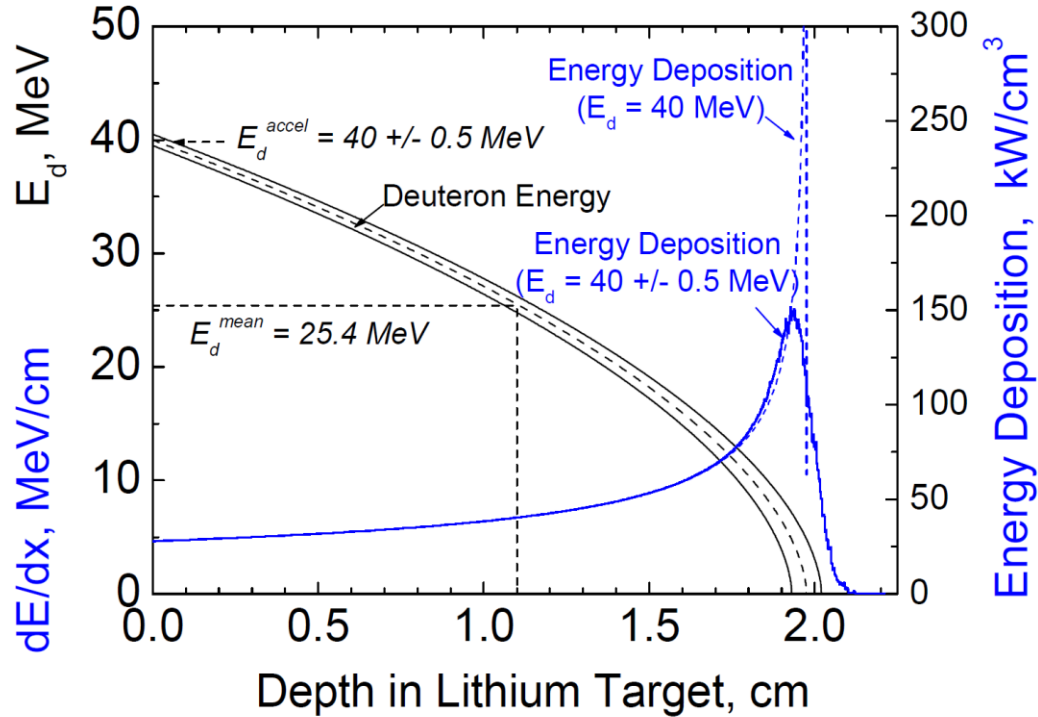
Photon flux (ph/cm²/s) at Target Assembly (TA)



- D+ ion beam stops in the lithium jet delivering a total power of 5 MW on a volume of $5 \times 20 \times 2.5 \text{ cm}^3$, with d-Li footprint area of $5 \times 20 \text{ cm}^2$.
- Deuterons lose their energy in Li by interactions with Li electron clouds and nuclei – all the processes have been taken into account in the MCNP6 energy deposition calculations with the TMESH card.
- For calculation of deuteron beam energy deposition in Li at the d-Li footprint area, **transport of neutrons, photons, deuterons, and protons** – 4 particles have been transported with the **MCNP6 mode n p d h**

MCNP6 horizontal cut of the D+ beam energy deposition at the d-Li footprint area with heat peak of 110 kW/cc





- Deuteron track length reach 2.1 cm;
- Peak Energy Deposition is 150 kW/cc at the depth of 2.0 cm (at the end of d-track)
- Average energy deposition in Li jet = $(40 \text{ MeV} \times 250 \text{ mA} = 10,000 \text{ kW}) / (20 \times 5 \times 2 \text{ cc}) = 50 \text{ kW/cc}$

Deuteron track depth (longitudinal, Ld) dependence on the D+ energy (Ed)



Ld=19.8 mm for Ed=40.0 MeV

Ld=20.3 mm for Ed=40.5 MeV

SRIM-2013.00

File Help, FAQ and Scientific Explanations

Help **Animate** Pause TRIM Change TRIM 100% ION ENERGY % Now: 803 of 99999 Ions

ION
 Ion Type H 2.0135 amu
 Ion Energy 40 MeV
 Ion Angle 0 degrees
 Completed 802 of 99999
 SHOW LIVE DATA HELP

TARGET DATA
 ? H (10) into DLayer 1 (1 layers, 1 atoms)

Layer Name	Width (A)	Density	Li (6.941)	Solid/Gas	Stop Corr.
1 DLayer 1	250000000	0.512	1.00000	Solid	1
Lattice Binding Energy			3		
Surface Binding Energy			1.67		

Calculation Parameters
 Backscattered Ions 0
 Transmitted Ions 0
 Vacancies/Ion 82.2

ION STATS
 Range Straggle
 Longitudinal 19.8 mm 196. um
 Lateral Proj. 150. um 222. um
 Radial 231. um 217. um

Type of Damage Calculation
 ? Quick: Kinchin-Pease
 Stopping Power Version
 ? SRIM-2008

% ENERGY LOSS

	Ions	Recoils
Ionization	99.95	0.02
Vacancies	0.00	0.00
Phonons	0.01	0.02

SPUTTERING YIELD

	Atoms/Ion	eV/Atom
TOTAL		
Li	0.000000	0.00

? Save every 10000 ions
 Random Number 1716356
 Counter HELP

Plots
 PLOT Window
 0 A - 250000000 A
 Max Target Depth 250000000

COLLISION PLOTS
 Ion/Recoils (XY) All
 Ion/Recoils (XZ) None
 Ions (no recoils) Tile
 Lateral View (YZ) Clear
 Background color White/Black

DISTRIBUTIONS
 Ion Distribution
 Ion/Recoil Distribution
 Lateral Range
 Ionization
 Phonons
 Energy to Recoils
 Damage Events
 Integral Sputtered
 Differential Ions

 Backscattered Ions
 Transmitted Ions
 Collision Details

3-D Plots 3D Help
 Ion Distribution 3D
 Recoil-Dist. 3D
 Ionization 3D
 Phonons 3D
 Target Damage 3D

5-XY Longitudinal
 + 12.5 mm
 Depth vs. Y-Axis
 -12.5 mm
 0 A - Target Depth - 25 mm
 Save Save As Print Label Clear

SRIM-2013.00

File Help, FAQ and Scientific Explanations

Help **Animate** Pause TRIM Change TRIM 100% ION ENERGY % Now: 719 of 99999 Ions

ION
 Ion Type H 2.0135 amu
 Ion Energy 40.5 MeV
 Ion Angle 0 degrees
 Completed 719 of 99999
 SHOW LIVE DATA HELP

TARGET DATA
 ? H (10) into DLayer 1 (1 layers, 1 atoms)

Layer Name	Width (A)	Density	Li (6.941)	Solid/Gas	Stop Corr.
1 DLayer 1	250000000	0.512	1.00000	Solid	1
Lattice Binding Energy			3		
Surface Binding Energy			1.67		

Calculation Parameters
 Backscattered Ions 0
 Transmitted Ions 0
 Vacancies/Ion 82.9

ION STATS
 Range Straggle
 Longitudinal 20.3 mm 223. um
 Lateral Proj. 154. um 226. um
 Radial 248. um 297. um

Type of Damage Calculation
 ? Quick: Kinchin-Pease
 Stopping Power Version
 ? SRIM-2008

% ENERGY LOSS

	Ions	Recoils
Ionization	99.95	0.03
Vacancies	0.00	0.00
Phonons	0.01	0.02

SPUTTERING YIELD

	Atoms/Ion	eV/Atom
TOTAL		
Li	0.000000	0.00

? Save every 10000 ions
 Random Number 1551224
 Counter HELP

Plots
 PLOT Window
 0 A - 250000000 A
 Max Target Depth 250000000

COLLISION PLOTS
 Ion/Recoils (XY) All
 Ion/Recoils (XZ) None
 Ions (no recoils) Tile
 Lateral View (YZ) Clear
 Background color White/Black

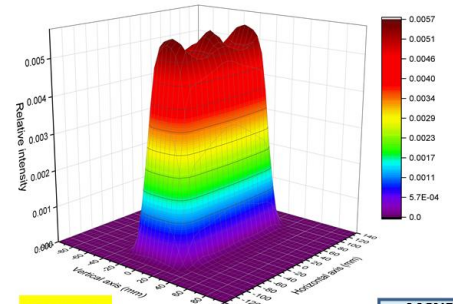
DISTRIBUTIONS
 Ion Distribution
 Ion/Recoil Distribution
 Lateral Range
 Ionization
 Phonons
 Energy to Recoils
 Damage Events
 Integral Sputtered
 Differential Ions

 Backscattered Ions
 Transmitted Ions
 Collision Details

3-D Plots 3D Help
 Ion Distribution 3D
 Recoil-Dist. 3D
 Ionization 3D
 Phonons 3D
 Target Damage 3D

5-XY Longitudinal
 + 12.5 mm
 Depth vs. Y-Axis
 -12.5 mm
 0 A - Target Depth - 25 mm
 Save Save As Print Label Clear

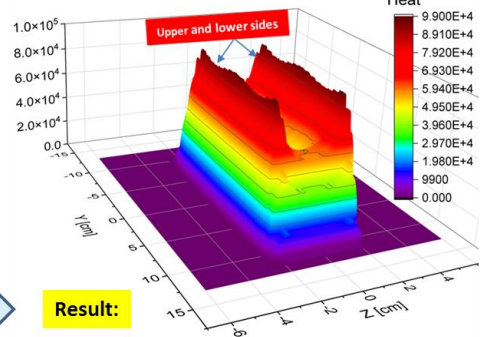
D+ beam profile (IFMIF/EVEDA)



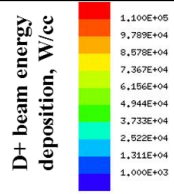
Source:

MCNP6 transport

Heat deposition in Li-Target plane section $x=-0.875$ cm

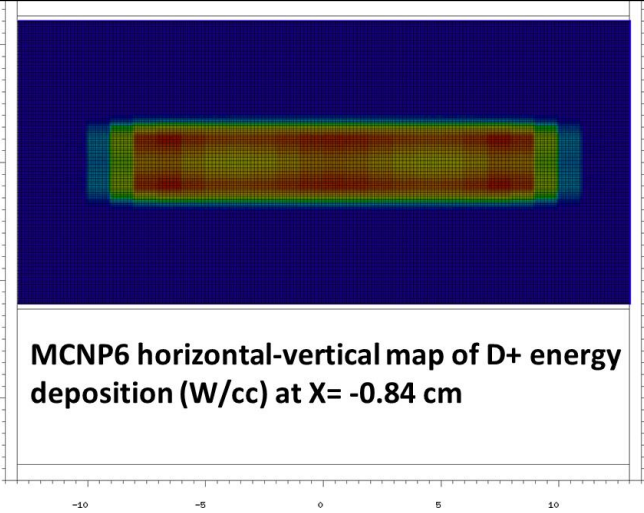


Result:

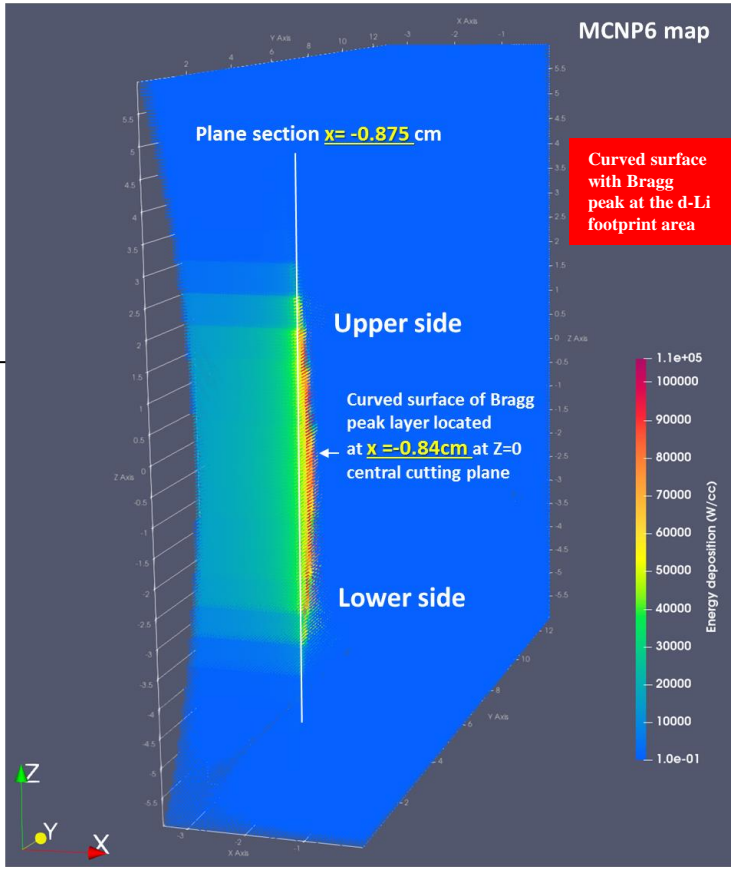


Heat at peak is 110 kW/cc
Integral D+ heat 4.8 MW

Notice: Bragg peak at Lithium thickness of **1.86 cm** corresponds to **-0.84 cm** coordinate in the MCNP model geometry with $X=0$ at the TA back plate and $X=-2.7$ cm is the front point of Li at $Z=0$ central plane: **1.86cm - 2.7cm = -0.84cm**



MCNP6 horizontal-vertical map of D+ energy deposition (W/cc) at $X = -0.84$ cm



MCNP6 map

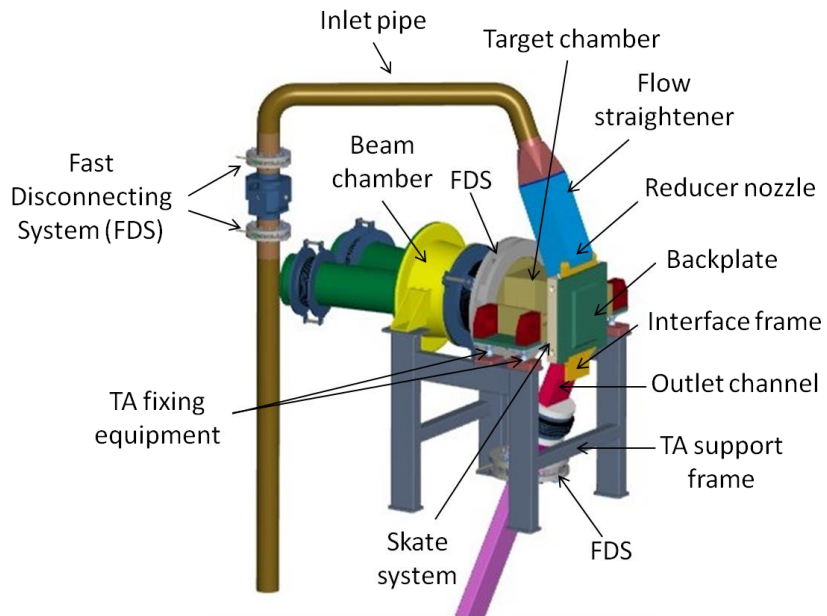
Curved surface with Bragg peak at the d-Li footprint area

Upper side

Curved surface of Bragg peak layer located at $x = -0.84$ cm at $Z=0$ central cutting plane

Lower side

Energy deposition (W/cc)



DONES Target Assembly (TA) components

```

PLOT WINDOW@r000u1105
04/25/22 18:24:58
IFMIF-DONES NB shutter,
lined-RBSB, md19,2,3,
KIT/INR, Apr,18 2022.
probid = 04/25/22 16:43:10
basis: XZ
( 1.000000, 0.000000, 0.000000)
( 0.000000, 0.000000, 1.000000)
origin:
( 0.00, 0.00, 0.00)
extent = ( 350.00, 350.00)

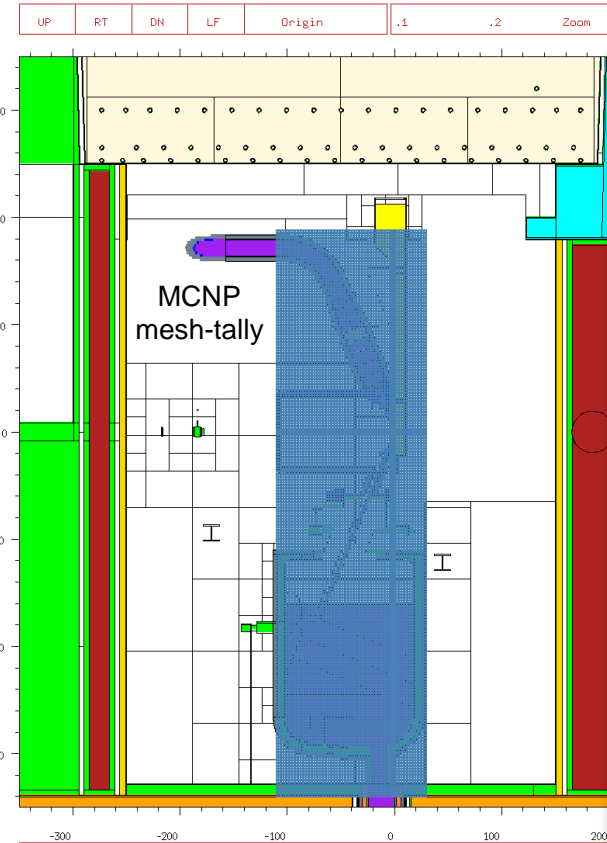
Mesh Tally 94

Value for cel 34179
in Cell 34179
xyz = 0.00, 0.00, 0.00

```

CURSOR	Restore	CellLine
PostScript	ROTATE	
COLOR	SCALES 1	LEVEL
XY	YZ	ZX
LABELS	L1 off	L2 off
HBODY on	FMESH 94	LEGEND off

[Click here on picture or menu](#)



MCNP model vertical cut of the DONES TA covered with mesh-tally

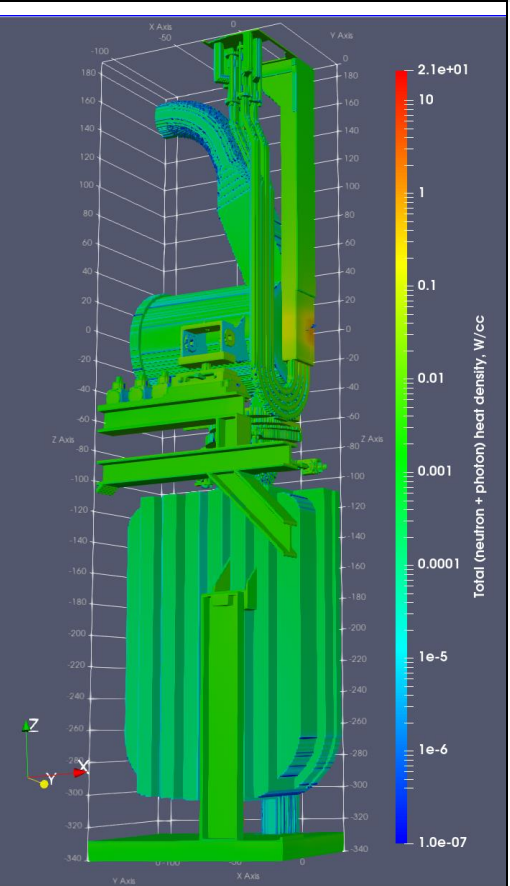
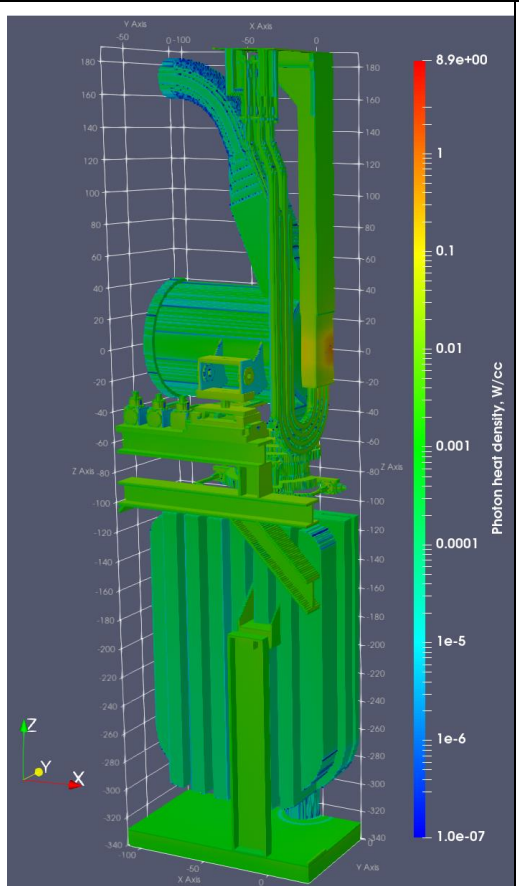
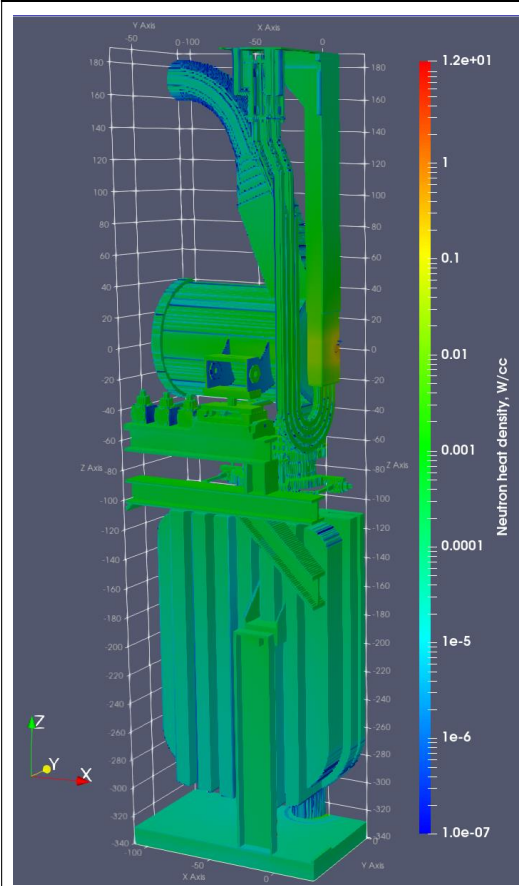


Fig. 1. Neutron heat density (W/cc) in actual materials of the MCNP model – look from the outside.

Fig. 2. Photon heat density (W/cc) in actual materials of the MCNP model – look from the outside.

Fig. 3. Total (neutron + photon) heat density (W/cc) in actual materials of the model – look from the outside.

TA materials:

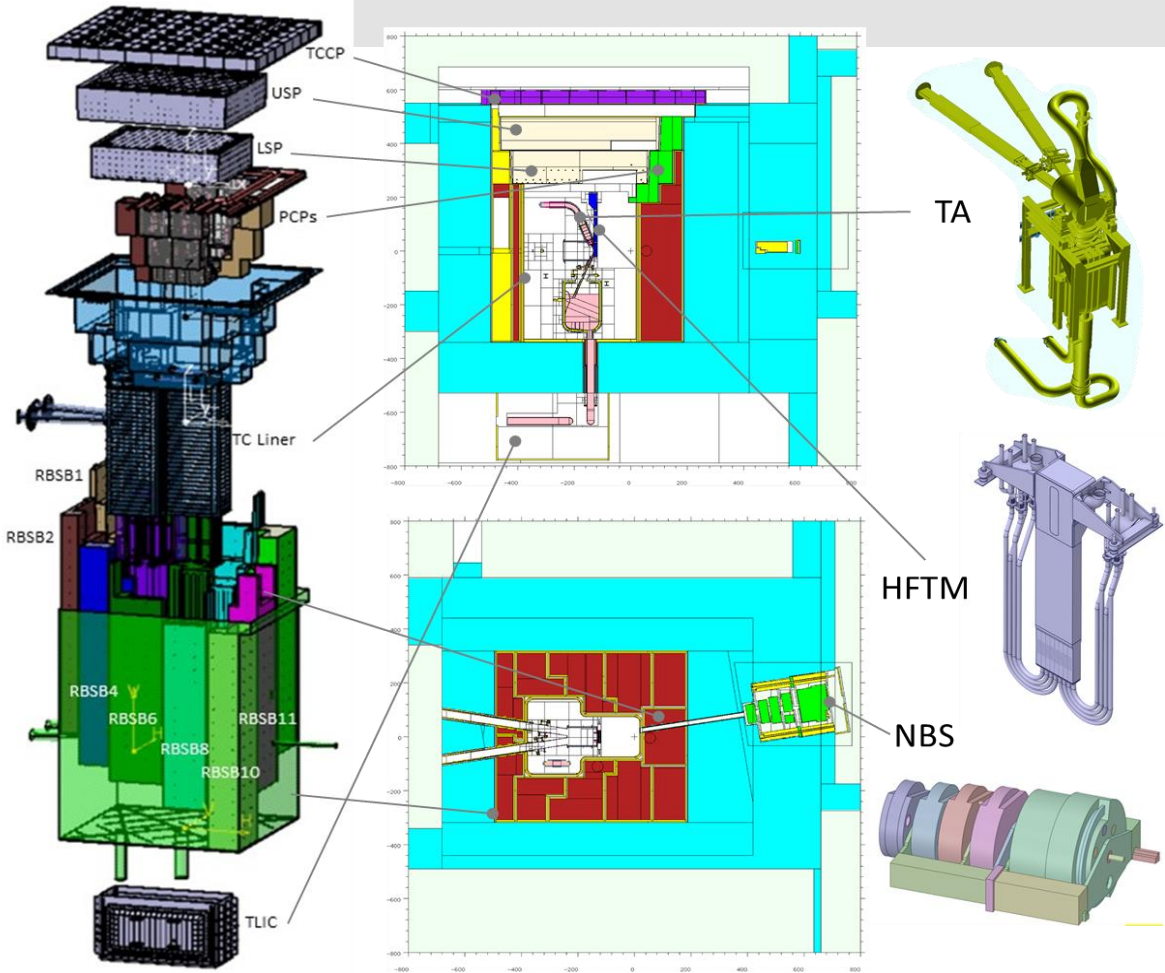
Steel SS316L material density 7.93 g/cc

EUROFER steel with density 7.87 g/cc

Lithium (Li) with impurities, its density is 0.512 g/cc.

Heating in Li jet at the area of deuteron footprint requires inclusion of the heat contributions of charged particles.

Power balance for the D+ beam energy released in Test Cell (TC) and internal components



Type of heat power	Heat, kW
Input heat power of D+ beam delivered by the IFMIF-DONES one accelerator beam current of I=125 mA	5000
D+ heat released in Li Target	4858.8
Neutron + Photon heat released in the TC components (numbered and displayed in the next slide):	
1) TC liner	15.2
2) Removable Biological Shielding Blocks	77.3
3) Bucket liner	0.2
4) Bucket	1.6
5) Piping and Cabling Plugs (PCP)	2.6
6) Lower Shielding Plug (LSP)	9.9
7) Upper Shielding Plug (USP)	0.01
The sum of 7 TC components:	107
Target Assembly (TA) structural parts	17.3
High Flux Test Module (HFTM)	16.9
Integral neutron and photon heat in all considered TC components	~141.2
Output: total integral heat released by D+, neutrons, and photons:	4858.8+141.2 = 5000



Test Cell Cover Plate (TCCP)

7 Heat 0.01 kW
Upper shielding plug (USP)

6 Heat 9.9 kW
Lower shielding plug (LSP)

5 Heat 2.6 kW
Pipeing & cabling plugs (PCP)

The sum of 7 Test Cell components is 107 kW

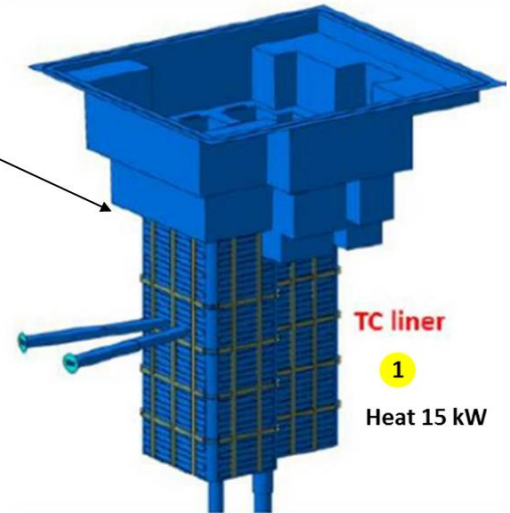
2 Heat 77 kW
Removable Biological Shielding Blocks (RBSB)

3 Heat 0.2 kW
Bucket liner

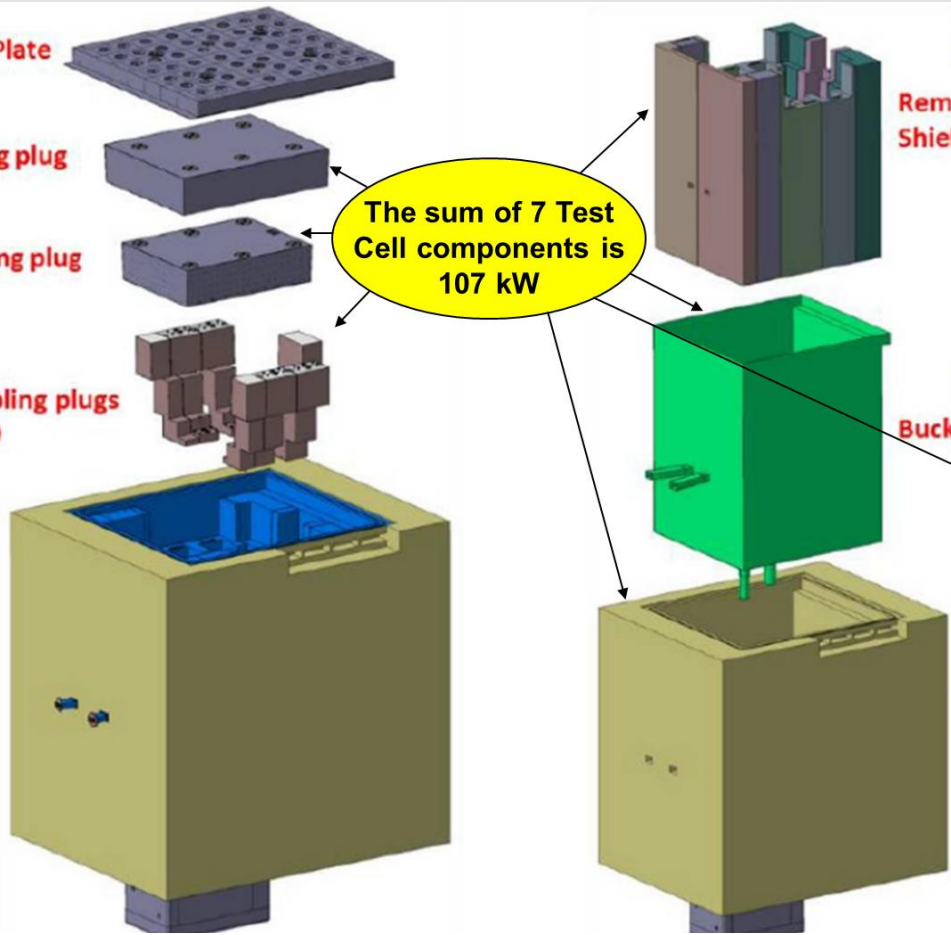
4 Heat 1.6 kW
Bucket



Heat 17.3 kW
Target system
+
HFTM
Heat 16.9 kW



TC liner
1
Heat 15 kW



- **The interactions of deuterons with lithium target** for the energies relevant to fusion applications, particularly $E_d=40$ MeV in IFMIF-DONES facility, are most accurately described with the McDeLicious code in its actual version McDeLicious-17, as an extension of the MCNP6.1 Monte Carlo radiation transport code. The McDeLicious code has been validated & verified in experimental and computational benchmarks.
- Using the D+ beam settings, McDeLicious samples neutrons and photons using evaluated d+ ${}^{6,7}\text{Li}$ data.
- The simple model of D+ interactions with cylindrical solid Li inside Aluminum capsule allows to investigate the D+ flux attenuation, track length, E_d attenuation, and D+ energy deposition. This work presented simple model with two settings of the D+ sources:
 - V1: Isotropic uniformly distributed disk
 - V2: Monodirectional directed source defined at a disk
- The (d-Li) reactions defined in McDeLicious-17 have been studied for the IFMIF-DONES facility. The beam of deuteron ions accelerated up to 40 MeV with current of 125 mA impinges the liquid Li target delivering 5 MW power. The presented results include distributions of D+ energy deposition, neutron and photon fluxes and heating.
- The integral heating calculations in IFMIF-DONES Test Cell (TC) components reveals that D+ energy deposition in liquid Li at thin Bragg peak with a D+ beam footprint area of $20 \times 5 \text{ cm}^2$ contributes 97% of total heating in the whole Test Cell volume. The 5 MW heat power of D+ beam delivered by the IFMIF-DONES is released by 97% in liquid lithium.

Backup slides:

Additional information about

- 1) McDeLicious code parallelization on Marconi-Fusion HPC**
- 2) Use of On-The-Fly (OTF) Monte Carlo variance reduction technique**

- **MCHIFI** (Monte Carlo High Fidelity) project has been organized for massively parallel computations on the EUROfusion Marconi-Fusion HPC for the most urgent and computationally demanded fusion neutronics tasks.
- The MCHIFI project was founded in 2012 to use the IFERC-CSC Helios supercomputer in the framework of the F4E Broader Approach (BA) to serve the ITER neutronics tasks.

- MCNP5 tested on the F4E Broader Approach IFERC-CSC Helios: 2x8 Intel Sandy-bridge EP processors with 2.7 Hz and 64 GB RAM per node:
 - Excellent scalability of MPI/OpenMP parallel runs of MCNP5 code up to 1024 cores in analogue runs, no variance reduction.
 - Speed-up equals ~450 on 512 cores, and ~850 of speed-up for 1024 cores.
 - OpenMP/MPI hybrid, the satisfactory speed-up of more than 2500 on 4096 cores was achieved for not-biased MCNP5 calculations, as it is illustrated in Figure 1

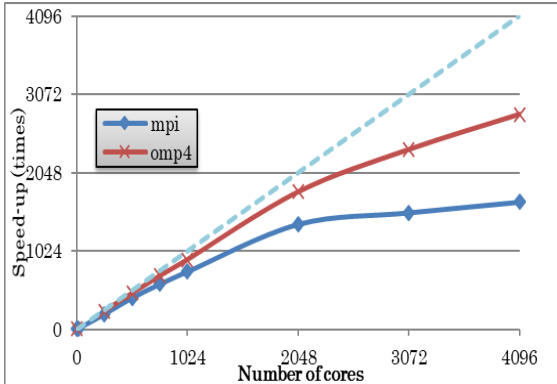


Figure 1. The MCNP5 speed-up on IFERC-CSC Helios supercomputer.

The optimal number of CPUs used in MCNP5/6 parallel calculations is dependent on complexity of the model. To improve the statistical errors of the MCNP5 results we are using the ADVANTG approach and the recently developed at KIT On-The-Fly (OTF) Monte Carlo variance reduction technique with dynamic Weight Window upper bounds, see Ref. [Yu Zheng, Y. Qiu, "Improvements of the on-the-fly MC variance reduction technique with dynamic WW upper bounds," *Nuclear Fusion* **62** (2022) 086036, <https://doi.org/10.1088/1741-4326/ac75fc>]

- McDeLicious tested on the EUROfusion HPC Marconi-Fusion with conventional partition (A3) based on INTEL Skylake with peak performance ~9.2 Pflops (2848 nodes). Each node is equipped with 2x24-cores Intel Xeon 8160 CPU (Skylake) at 2.10 GHz and 192 GB of RAM per node.
- Speed-up MPI-parallel performance has been measured and presented in Figure 2 for the McDeLicious code for IFMIF-DONES radiation deep-penetration shielding tasks with variance reduction.

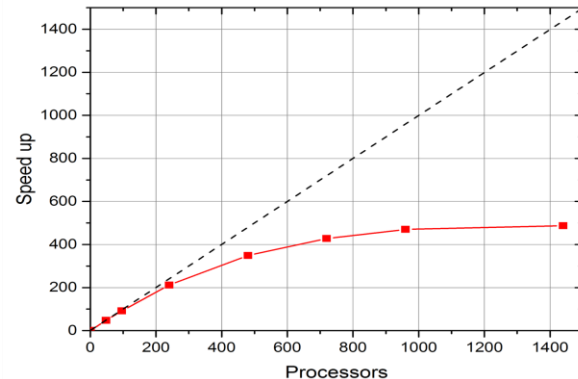
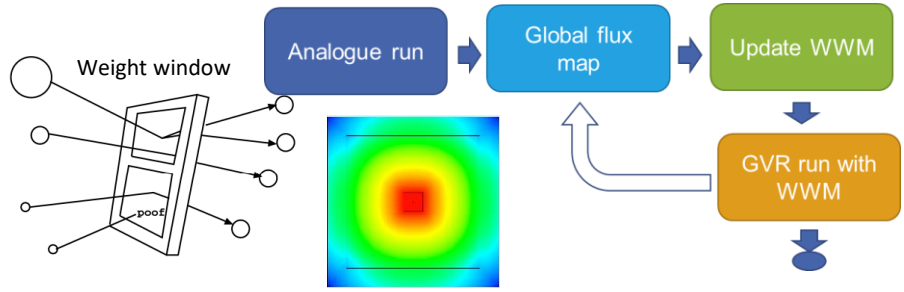


Figure 2. The speed-up of McDeLicious code on Marconi-Fusion HPC.

- **OTF-GVR: On-The-Fly Global Variance Reduction**
- Weight windows mesh (WWM) is a common method used for MC shielding calculation.
- Performs “on-the-fly” iterations to get a global flux map and a weight-window mesh.
- Using novel dynamic WW upper bound method to solve the neutron streaming and “long-history” particles
- Comparing with ADVANTG, OTF-GVR shows **enhancement by a factor of 20**



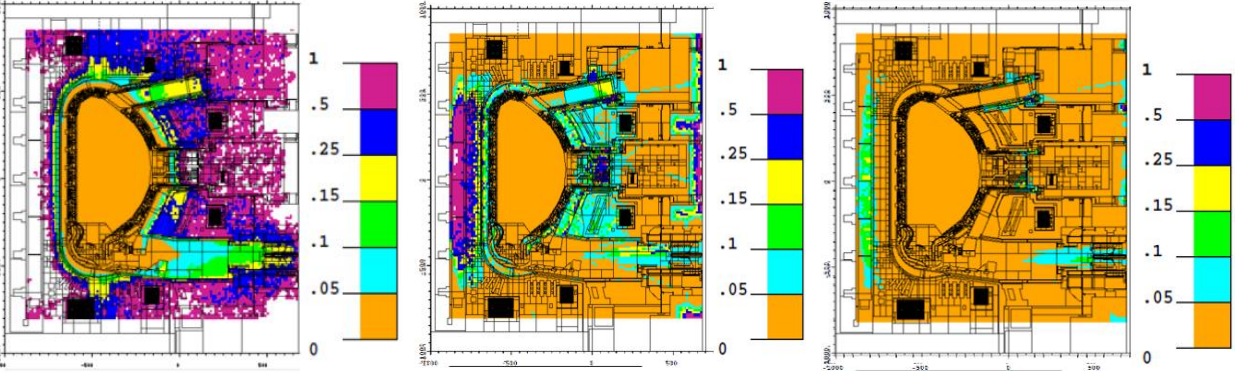
OTF-GVR:

$$n(\vec{r}) \approx m(\vec{r})\bar{w}(\vec{r})$$

$$\bar{w}(\vec{r}) = \phi(\vec{r})/\max(\phi(\vec{r})) \Rightarrow \bar{w}(\vec{r}) = c \times \phi(\vec{r})/\max(\phi(\vec{r}))$$

Definition of “c” to avoid “long-history” by limiting the splitting in the OTF run in Ref. [Yu Zheng, Yuefeng Qiu, et al., “An improved on-the-fly global variance reduction technique by automatically updating weight window values for Monte Carlo shielding calculation”, Fusion Eng. Des. 147 (2019) 111238, <https://doi.org/10.1016/j.fusengdes.2019.06.011>]

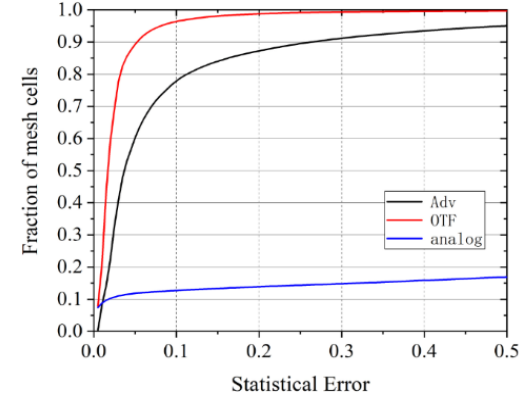
On-the-fly Global weight window mesh generation



Analogue run

ADVANTG WWM run

OTF-GVR run



Percentage of mesh cells and rel. error

Ref.: [Yu Zheng et al 2022 Nucl. Fusion 62 086036, <https://doi.org/10.1088/1741-4326/ac75fc>]